only to the particular shield configuration investigated. It is interesting to observe whether or not theoretical expressions can be found that are approximately proportional to the data. However, this approach does not permit one to calculate the radiation attenuation for a different shield and duct arrangement.

In summary, the paper by Nilsson and Sandlin represents a disappointing attempt to use source-separation duct analysis for the following reasons:

1) Measured quantities did not closely approximate defined quantities.

2) The experimental uncertainty associated with some of the separated flux component data was too great to be useful in detailed theoretical analysis.

3) Theoretical correlations include undetermined multiplying constants.

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An Acknowledgment

In two recent papers I have discussed a method of modifying the one-group spherical harmonics method through the introduction of a flexible truncation scheme. In the first paper[1] only slab geometry was considered, and three examples of the truncation scheme were given, one of which led to a set of P-N-like equations with the property that for all $N \geq 1$ one pair of the exponential solutions in a homogeneous medium has the exact asymptotic transport-theory exponent. In a subsequent paper[2] this asymptotic truncation procedure was extended to an arbitrary, three-dimensional geometry.

Recently, I came across a set of lecture notes by B. Davison, dated 1947, in which the same asymptotic truncation procedure was suggested[3]. While Davison only considered homogeneous systems in slab geometry, and did not discuss the associated vacuum boundary conditions or the interface boundary conditions in a multilayered system, it must be acknowledged that the basic asymptotic truncation idea was suggested by Davison almost 20 years before my papers on the subject were written.

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[1]G. C. POMRANING, "A Generalized P-N Approximation for Neutron Transport Problems," *Nucleonik*, **6**, 348 (1965).

[2]G. C. POMRANING, "An Asymptotically Correct Approximation to the Multidimensional Transport Equation," *Nucl. Sci. Eng.*, **22**, 328 (1965).

[3]B. DAVISON, *Transport Theory of Neutrons*, Report LT-18, 124-129, National Research Council of Canada (1947).