

## Effect of Bragg Cutoff on the Diffusion of Thermal Neutrons in a Finite Solid Medium

In an earlier paper (1) having the above title some numerical errors in the values of average energy transfer cross section  $g^{12}$  and  $g^{21}$  for beryllium at 100°K have been detected, which affect considerably the parameters  $B_1^2$ ,  $B_2^2$ ,  $A'/A$ ,  $C'/C$ , and the general results obtained. Therefore a recalculation of the problem with corrected constants was necessary. This was done for a beryllium slab of thickness 198.72 cm at a temperature of 100°K. An improvement in the formula for the average diffusion constant was also introduced. Instead of  $D^i = \frac{1}{3} \lambda_{tr}^i$  ( $i = 1, 2$ ) used previously, the more accurate expression

$$D^i = \frac{\lambda_{tr}^i}{3(1 + \sum_a \lambda_{tr}^i)}$$

was now introduced. The changes in the average values of the nuclear constants and parameters are given in Tables I and II. The ratio of the neutron fluxes in the two groups and the mean energy of neutrons as a function of  $x$  are given in Fig. 1. Both these quantities are now quite different from what was published before. In fact both  $\phi^1(x)/\phi^2(x)$  and  $\bar{E}(x)$  tend to attain a nearly constant value for the lower temperature (100°K) of the medium before showing a slight rise for  $\phi^1/\phi^2$  and consequently a small decrease for  $\bar{E}(x)$  near the boundary, which is the effect of the finite size of the slab.

TABLE I

Constants	Values used in ref. 1	Revised values
$D^1$ (cm)	46.08	30.07
$D^2$ (cm)	0.4072	0.4364
$g^{12}$ (cm <sup>-1</sup> )	$6.6671 \times 10^{-7}$	$3.9770 \times 10^{-4}$
$g^{21}$ (cm <sup>-1</sup> )	$4.8024 \times 10^{-6}$	$2.8621 \times 10^{-3}$

TABLE II

Parameters depending on constants in Table I	Values used in ref. 1	Revised values
$B_1^2$ (cm <sup>-2</sup> )	$4.0051 \times 10^{-5}$	$5.0090 \times 10^{-3}$
$B_2^2$ (cm <sup>-2</sup> )	$8.3691 \times 10^{-5}$	$2.0381 \times 10^{-4}$
$A'/A$	$-2.7103 \times 10^5$	-361.84
$C'/C$	$3.0076 \times 10^{-3}$	1.4716

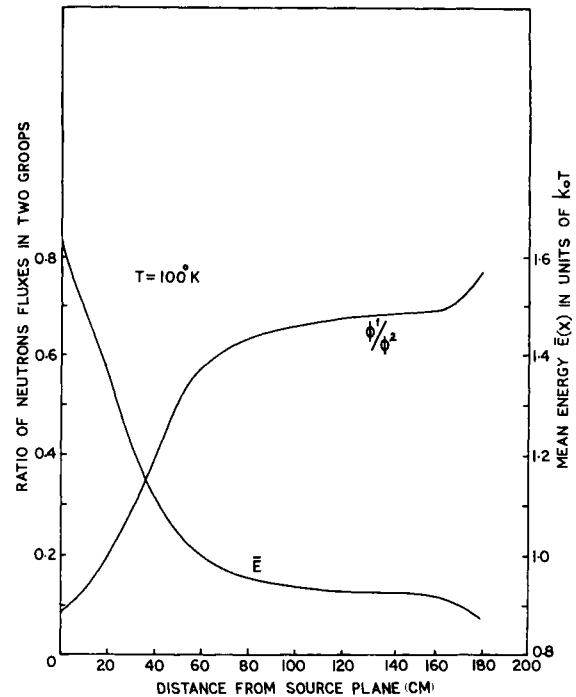


FIG. 1. The ratio of neutron fluxes in the two groups and the mean neutron energy in a 198.72 cm thick beryllium slab at temperature  $T = 100^\circ\text{K}$  plotted as a function of distance from the source.

When the corrected values of  $g^{12}$  and  $g^{21}$  are used for calculating the equilibrium flux distribution in a semi-infinite slab, as done by Jain and Lawande (2), their results for beryllium at 100°K are also completely changed. For the asymptotic value of the ratio of cold to thermal neutron flux, one now obtains

$$\left. \frac{\phi_{\text{cold}}}{\phi_{\text{thermal}}} \right|_{\text{asym.}} = 0.68 = \frac{C'}{C}$$

instead of 333 quoted by them. Results for beryllium at 300°K are unaltered.

## REFERENCES

1. S. SANATANI AND L. S. KOTHARI, *Nuclear Sci. and Eng.* **11**, 211-217 (1961).
2. R. D. JAIN AND S. V. LAWANDE, *Nuclear Sci. and Eng.* **11**, 228-9 (1961).

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