Corrigendum

K. OISHI, Y. IKEDA, H. MAEKAWA, and T. NAKAMURA, "Experiment and Analysis of Neutron Spectra in a Concrete Assembly Bombarded by 14-MeV Neutrons," *Nucl. Sci. Eng.*, **103**, 46 (1989).

Table I is to be replaced by the following:

TABLE I

Components of the Concrete Assembly

Component	Atomic Density	Estimated Error (%)
Silicon	1.0391+22[a]	(±2)
Aluminum	2.2527+21	(±2)
Iron	6.4302+20	(±2)
Calcium	4.7704+21	(±2)
Magnesium	4.8429+20	(±2)
Sulfur	1.0688+20	(±2)
Sodium	5.9966+20	(±2)
Potassium	4.0024+20	(±2)
Titanium	6.3584+19	(±2)
Phosphorus	2.2605+19	(±2)
Manganese	2.8270+19	(±2)
Barium	1.0463+19	(±2)
Vanadium	4.7765+18	(±10)
Cobalt	1.9736+17	(±10)
Zinc	1.3087+18	(±10)
Copper	6.1017+17	(±10)
Nickel	2.7330+17	(±10)
Carbon	1.9442+20	(±2)
Hydrogen	1.7523+22	(±2)
Oxygen	4.0574+22	(±2)

[a]Read as 1.0391×10^{22}.

Figure 7 is to be replaced by the following:

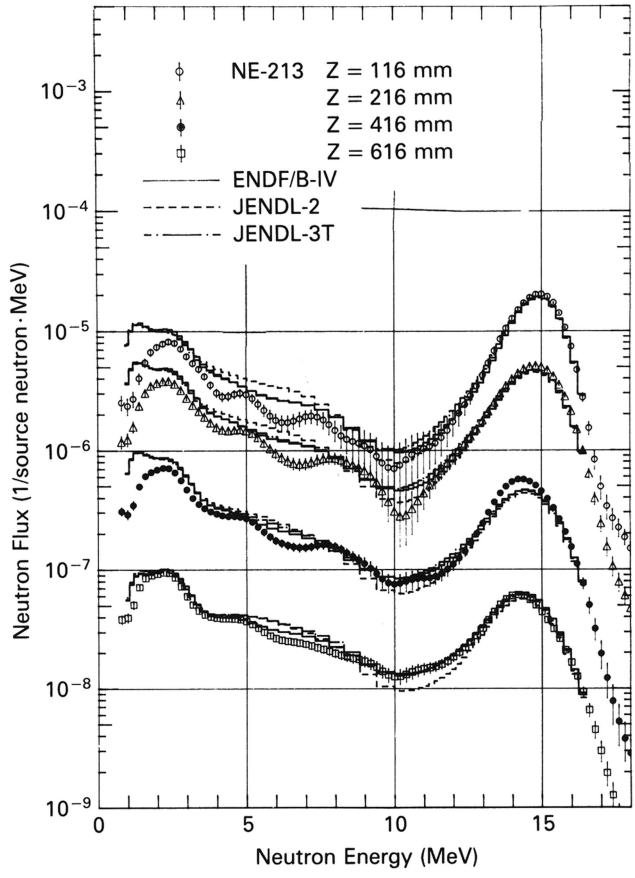

Fig. 7. Comparisons between calculated and experimental spectra measured by the NE-213 spectrometer.

Table VI is to be replaced by the following:

TABLE VI

C/E Values for Various Reaction Rates

Position (mm)	Experiment	C/E (ENDF/B-IV)	C/E (JENDL-2)	C/E (JENDL-3T)	Position (mm)	Experiment	C/E (ENDF/B-IV)	C/E (JENDL-2)	C/E (JENDL-3T)
$^{90}Zr(n,2n)^{89}Zr$ Reaction					$^{58}Ni(n,2n)^{57}Ni$ Reaction				
0	1.752−28[a]	0.99	0.99	0.99	0	9.372−30	0.96	0.95	0.95
25	1.241−28	0.98	0.98	0.98	25	6.620−30	0.93	0.93	0.94
50	8.761−29	0.96	0.96	0.97	50	4.614−30	0.92	0.92	0.93
100	4.463−29	0.94	0.93	0.95	100	2.327−30	0.90	0.89	0.90
200	1.227−29	0.94	0.92	0.95	200	6.266−31	0.90	0.87	0.90
400	1.200−30	0.94	0.89	0.94	400	6.228−32	0.85	0.80	0.84
550	2.384−31	0.91	0.83	0.89	550	1.104−32	0.90	0.82	0.87
$^{93}Nb(n,2n)^{92m}Nb$ Reaction					$^{27}Al(n,\alpha)^{24}Na$ Reaction				
0	9.816−29	0.99	0.99	0.99	0	2.537−29	0.96	0.96	0.96
25	7.196−29	0.93	0.93	0.95	25	1.879−29	0.96	0.96	0.98
50	5.354−29	0.93	0.94	0.95	50	1.363−29	0.95	0.95	0.97
100	2.862−29	0.91	0.90	0.93	100	7.564−30	0.91	0.91	0.94
200	8.518−30	0.90	0.88	0.93	200	2.377−30	0.90	0.87	0.93
400	9.343−31	0.91	0.84	0.92	400	2.716−31	0.91	0.84	0.93
550	1.904−31	0.91	0.81	0.90	550	5.698−32	0.92	0.81	0.92
$^{56}Fe(n,p)^{56}Mn$ Reaction					$^{58}Ni(n,p)^{58}Co$ Reaction				
0	2.258−29	0.95	0.95	0.95	0	7.227−29	0.94	0.96	0.94
25	1.701−29	0.93	0.94	0.95	25	5.994−29	0.97	1.02	0.99
50	1.219−29	0.94	0.95	0.97	50	4.806−29	0.97	1.01	0.99
100	6.747−30	0.91	0.91	0.94	100	3.010−29	0.96	1.00	0.98
200	2.127−30	0.89	0.88	0.92	200	1.154−29	0.95	0.97	0.97
400	2.452−31	0.91	0.85	0.93	400	1.732−30	0.94	0.93	0.97
550	5.158−32	0.92	0.82	0.92	550	3.820−31	0.98	0.94	1.01
$^{115}In(n,n')^{115m}In$ Reaction					$^{197}Au(n,\gamma)^{198}Au$ Reaction				
0	2.709−29	0.92	0.97	0.91	0	1.328−27	1.00	1.00	1.00
25	2.707−29	0.98	1.03	0.97	25	2.919−27	1.04	1.05	1.05
50	2.233−29	1.05	1.10	1.04	50	4.238−27	0.99	1.00	1.00
100	1.624−29	1.02	1.06	1.01	100	5.971−27	0.88	0.90	0.89
200	7.102−30	1.02	1.05	1.02	200	4.932−27	0.83	0.88	0.86
400	1.164−30	1.05	1.05	1.06	400	1.196−27	0.88	0.97	0.95
550	2.678−31	1.07	1.05	1.10	550	2.356−28	0.93	1.04	1.03

[a]Read as 1.752×10^{-28}.

Figures 8a, 8b, and 8c are to be replaced by the following:

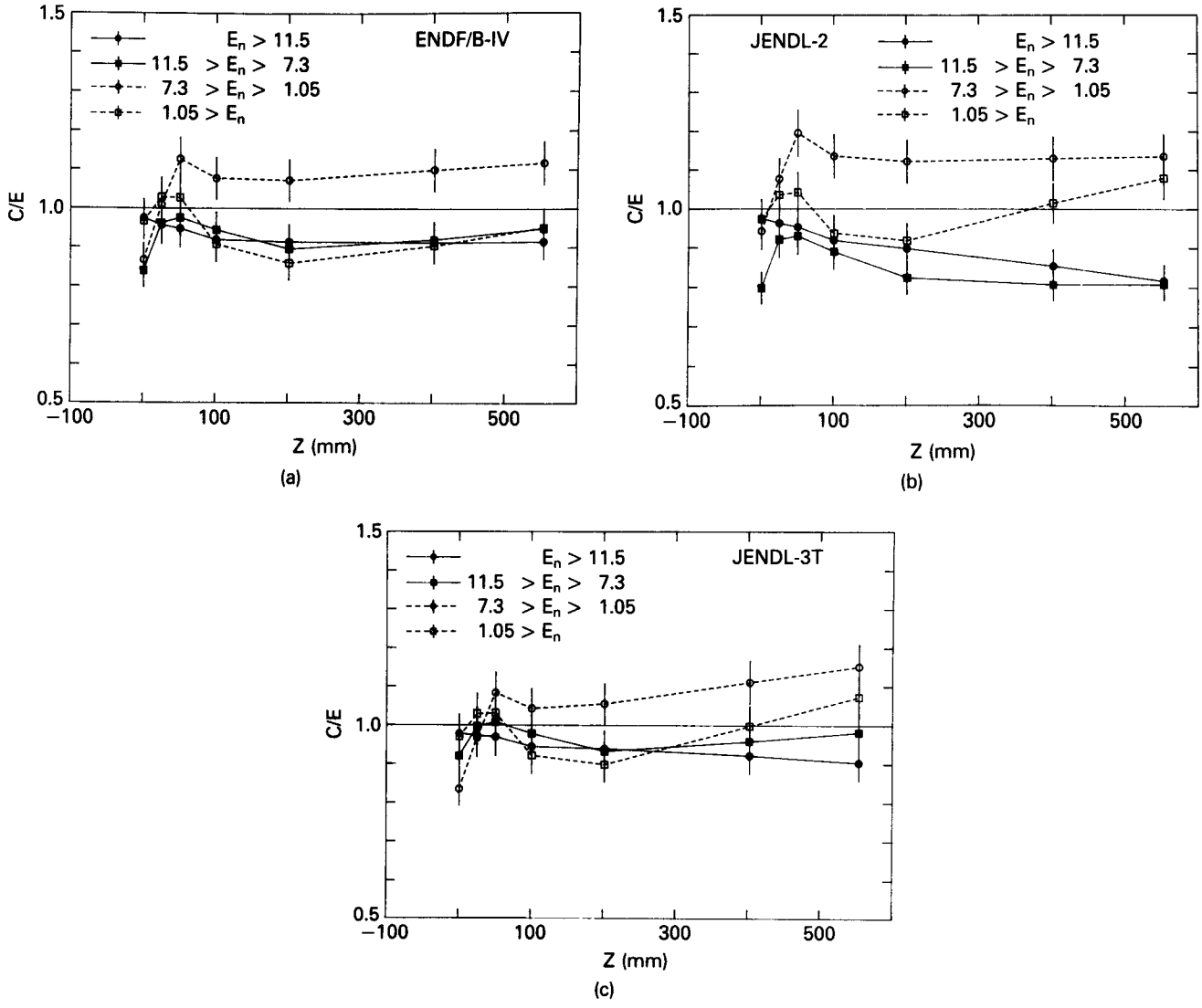

Fig. 8. The C/E values of integrated neutron flux for several neutron energy ranges calculated with (a) ENDF/B-IV, (b) JENDL-2, and (c) JENDL-3T.

The authors deeply regret any misunderstandings the original incorrect figures and tables may have caused.